ANSI/ANS-58.8-2019



# Time Response Criteria for Manual Actions at Nuclear Power Plants

ANSI/ANS-58.8-2019



An American National Standard

Published by the American Nuclear Society 555 N. Kensington Ave La Grange Park, IL 60526 American National Standard Time Response Criteria for Manual Actions at Nuclear Power Plants

Secretariat American Nuclear Society

Prepared by the American Nuclear Society Standards Committee Working Group ANS-58.8

Published by the American Nuclear Society 555 North Kensington Avenue La Grange Park, Illinois 60526 USA

Approved August 8, 2019 by the American National Standards Institute, Inc.

### American National Standard

Designation of this document as an American National Standard attests that the principles of openness and due process have been followed in the approval procedure and that a consensus of those directly and materially affected by the standard has been achieved.

This standard was developed under the procedures of the Standards Committee of the American Nuclear Society; these procedures are accredited by the American National Standards Institute, Inc., as meeting the criteria for American National Standards. The consensus committee that approved the standard was balanced to ensure that competent, concerned, and varied interests have had an opportunity to participate.

An American National Standard is intended to aid industry, consumers, governmental agencies, and general interest groups. Its use is entirely voluntary. The existence of an American National Standard, in and of itself, does not preclude anyone from manufacturing, marketing, purchasing, or using products, processes, or procedures not conforming to the standard.

By publication of this standard, the American Nuclear Society does not insure anyone utilizing the standard against liability allegedly arising from or after its use. The content of this standard reflects acceptable practice at the time of its approval and publication. Changes, if any, occurring through developments in the state of the art, may be considered at the time that the standard is subjected to periodic review. It may be reaffirmed, revised, or withdrawn at any time in accordance with established procedures. Users of this standard are cautioned to determine the validity of copies in their possession and to establish that they are of the latest issue.

The American Nuclear Society accepts no responsibility for interpretations of this standard made by any individual or by any ad hoc group of individuals. Inquiries about requirements, recommendations, and/or permissive statements (i.e., "shall," "should," and "may," respectively) should be sent to the Publications and Standards Department at Society Headquarters. Action will be taken to provide appropriate response in accordance with established procedures that ensure consensus.

Comments on this standard are encouraged and should be sent to Society Headquarters.

Published by

American Nuclear Society 555 North Kensington Avenue La Grange Park, Illinois 60526 USA



Copyright © 2019 by American Nuclear Society. All rights reserved.

Any part of this standard may be quoted. Credit lines should read "Extracted from American National Standard ANSI/ANS-58.8-2019 with permission of the publisher, the American Nuclear Society." Reproduction prohibited under copyright convention unless written permission is granted by the American Nuclear Society.

Printed in the United States of America

**Inquiry Requests** The American Nuclear Society (ANS) Standards Committee will provide responses to inquiries about requirements, recommendations, and/or permissive statements (i.e., "shall," "should," and "may," respectively) in American National Standards that are developed and approved by ANS. Responses to inquiries will be provided according to the Policy Manual for the ANS Standards Committee. Nonrelevant inquiries or those concerning unrelated subjects will be returned with appropriate explanation. ANS does not develop case interpretations of requirements in a standard that are applicable to a specific design, operation, facility, or other unique situation only and therefore is not intended for generic application.

Responses to inquiries on standards are published in ANS's magazine, *Nuclear News*, and are available publicly on the ANS website or by contacting the Publications and Standards Department.

Inquiry Format

Inquiry requests shall include the following:

- (1) the name, company name if applicable, mailing address, and telephone number of the inquirer;
- (2) reference to the applicable standard edition, section, paragraph, figure, and/or table;
- (3) the purpose(s) of the inquiry;
- (4) the inquiry stated in a clear, concise manner;
- (5) a proposed reply, if the inquirer is in a position to offer one.

Inquiries should be addressed to

American Nuclear Society Publications and Standards Department 555 N. Kensington Avenue La Grange Park, IL 60526

or standards@ans.org

#### **Foreword** (This foreword is not a part of American National Standard "Time Response Criteria for Manual Actions at Nuclear Power Plants" ANSI/ANS-58 8-2019 but is included for informational purposes only.)

at Nuclear Power Plants," ANSI/ANS-58.8-2019, but is included for informational purposes only.)

In 2015, members of the Institute of Electrical and Electronics Engineers' Power and Energy Society, Nuclear Power Engineering Committee, Subcommittee 5 (SC5), Human Factors, Control Facilities, and Human Reliability, reviewed the previous issue of this standard [ANS-58.8-1994 (R2017)]. There was consensus among SC5 members that the standard did not fully address the change in estimation of time required for manual actions due to the changes in technology and operations since the last revision of the standard. For example, there was a commonly shared concern that the empirical basis for the previous issue of this standard was drawn from simulator studies conducted in analog control rooms, and therefore, the standard lacked an empirical basis for application to highly integrated digital control rooms. Additionally, members were concerned that the data upon which the standard was developed were not publicly available, and therefore, the validity of the time constants derived from those studies was not scrutable.

Simply updating the empirical basis and related time constants of the standard was not adopted in this revision of the standard, ANSI/ANS-58.8-2019, because, given the design flexibility of digital technology as well as its rapidly evolving nature, such an approach would result in questions about whether the simulated control rooms and plant models used to develop the basis for the revised standard were representative of the designs to which the standard would be applied and whether the standard could quickly become outdated.

This revision of the standard attempts to address the concerns mentioned above. Technology- and operation-neutral criteria and methods were adopted to determine time requirements for manual actions. This revision differs in the following aspects:

- (1) The previous version of this standard provided a calculational approach for determining acceptable manual action times. The industry has shifted to a performance-based approach using the plant and simulators to demonstrate achievable manual action times. The revised approach used in this standard allows preliminary time determinations by calculation; however, actual plant or simulator performance verification is required when feasible.
- (2) The scope of the previous revision of this standard was limited to safety-related operator actions to be used in the design of light water reactor nuclear power plants. This revision is applicable to all nuclear power plants. Additionally, as mentioned above, it provides guidance not only for calculating manual action times during design, but also for validation of the calculated manual action times as well as for continued verification and configuration control of the times for operating plants. Owners and designers for other types of nuclear facilities might find portions of this standard helpful in determining the acceptability of manual actions for their facilities.
- (3) The term "safety-related operator action" is not used in this revision. The term "time-critical action" (TCA) is used instead and refers to a manual action or series of actions, the performance of which within specified time constraints has been credited in the plant's design analysis (e.g., safety analysis) or licensing basis.

The criteria contained in this standard establish timing requirements for manual actions with specific time constraints associated with design-basis events (DBEs) and for manual actions to meet the plant licensing or authorization basis for nuclear power plants. These criteria are used to determine whether an activity can be initiated by operator action or requires automatic initiation. The time response criteria given in this standard adopt time intervals and other restrictions to ensure that adequate safety margins are applied to system and plant design and safety evaluations. Requirements and guidance for design of associated instrumentation, controls, indicators, and annunciators necessary for TCAs are provided. In addition, requirements and guidance for validation of the manual action times are also provided.

The scope of this standard is limited to manual actions associated with those DBEs that are required to be analyzed in the licensing-basis documentation of nuclear power plants. Should the scope of the licensing-basis documentation be expanded, this standard should be applied to new DBEs that require operator actions with specific time constraints.

Other actions that have a time limit that are not part of the licensing-basis documentation are not included in this standard. This standard is not intended to supersede time criteria included in preexisting licensee regulatory commitments but may be used to help justify changes to such commitments.

The recommendations in the Pressurized Water Reactor Owner's Group document PWROG-16030-NP, "Time Critical Action/Time Sensitive Action Program Standard," Revision 1, were considered in this standard, and; therefore, is relatively consistent with these recommendations.

NUREG/BR-0303, "Guidance for Performance-Based Regulation," was reviewed by the ANS-58.8 Working Group, and this standard was revised to clarify that performance-based approaches consistent with NUREG/BR-0303 are allowed.

This standard might reference documents and other standards that have been superseded or withdrawn at the time the standard is applied. A statement has been included in the references section that provides guidance on the use of references.

This standard was prepared by the ANS-58.8 Working Group of the American Nuclear Society. The following members contributed to this standard:

H. Liao (Chair), JENSEN HUGHES

L. T. Christensen, Bechtel Power Corporation
D. R. Desaulniers, U.S. Nuclear Regulatory Commission
J. D. Ford, AREVA
R. B. Fuld, Westinghouse
M. Ravan, AREVA
S. Sallade, Exelon Corporation
S. L. Stamm, Individual
R. E. Vail, AECOM
M. Weiner, Duke Energy

The Simulators, Instrumentation, Control Systems, Software & Testing Subcommittee had the following membership at the time of its approval of this standard:

P. K. Guha (Chair), U.S. Department of Energy L. T. Christensen (Vice Chair), Bechtel Power Corporation

J. K. August, *Southern Company* S. M. Cetiner, *Oak Ridge National Laboratory*  J. B. Florence, Nebraska Public Power District
J. P. Glover, Graftel, Inc.
H. Liao, Jensen Hughes
E. M. Lloyd, Exitech Corporation
J. A. Sickle, Exelon Corporation
K. Singh, EDF Energy
M. E. Smith, Nuclear Innovation North America, LLC
B. Stevens, Exelon Corporation

The Large Light Water Reactor Consensus Committee (LLWRCC) had the following membership at the time of its approval of this standard:

C. E. Carpenter (Chair), U.S. Department of Energy W. B. Reuland (Vice Chair), Individual

R. E. Becse, Westinghouse Electric Company R. J. Burg, Engineering Planning & Management, Inc. L. T. Christensen, Bechtel Power Corporation M. J. Colby, Global Nuclear Fuel J. B. Florence, Nebraska Public Power District M. L. French, WECTEC D. Gardner, Kairos Power S. W. Gebers, Quantum Nuclear Services J. P. Glover, Graftel, Inc. P. K. Guha, U.S. Department of Energy E. M. Johnson-Turnipseed, Entergy Corporation M. A. Linn, Oak Ridge National Laboratory E. M. Lloyd, Exitech Corporation R. Markovich, Contingency Management Consulting T. K. Meneely, Westinghouse Electric Company, LLC C. H. Moseley, Jr., ASME NQA Liaison S. D. Routh, Bechtel Power Corporation S. L. Stamm, Individual

LLWRCC Observers:

J. M. Bonfiglio, Framatome Inc.

R. M. Ruby, Individual

J. C. Saldarini, Advanced Reactor Concepts, LLC

# Contents

### Section

## Page

1	Introduction       1         1.1       Scope       1         1.2       Purpose       1         1.3       Applications       1         1.4       Assumptions       1         1.5       Exclusions       2
2	Acronyms and definitions.22.1Acronyms22.2Shall, should, and may22.3Definitions.3
3	General requirements.43.1TCA sources.43.2Effect of operator error.43.3TCA $\leq$ 30 minutes.43.4Manual actions outside the control room43.5Instrumentation.53.6Performance time requirements.5
4	Calculation of manual action time intervals during design
5	Validation of manual action times prior to operation
6	Periodic revalidation
7	Administrative control programs
8	TCA program
9	References